



# ***Nuclear Energy for Space Exploration***

**Presented**

***June 5-6, 2010***

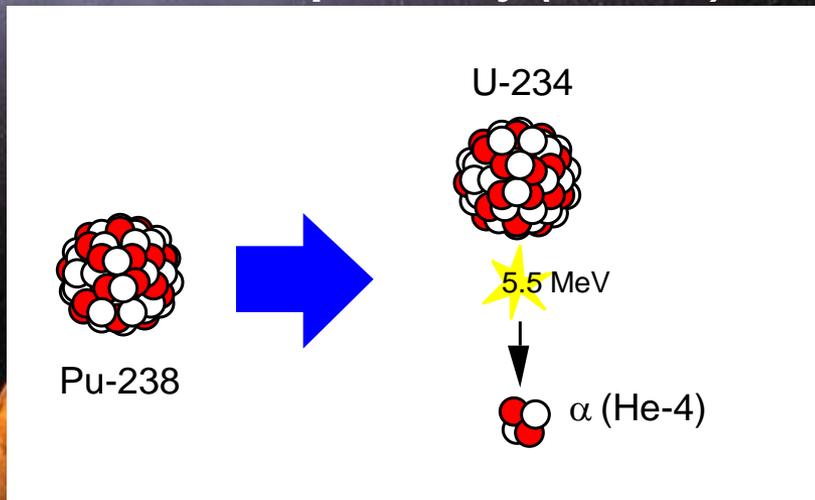
***Dr. Michael G. Houts  
Marshall Space Flight Center***

***NASA Speakers Bureau***



# Basics of Space Nuclear Systems

## Radioisotope Decay (Pu-238)

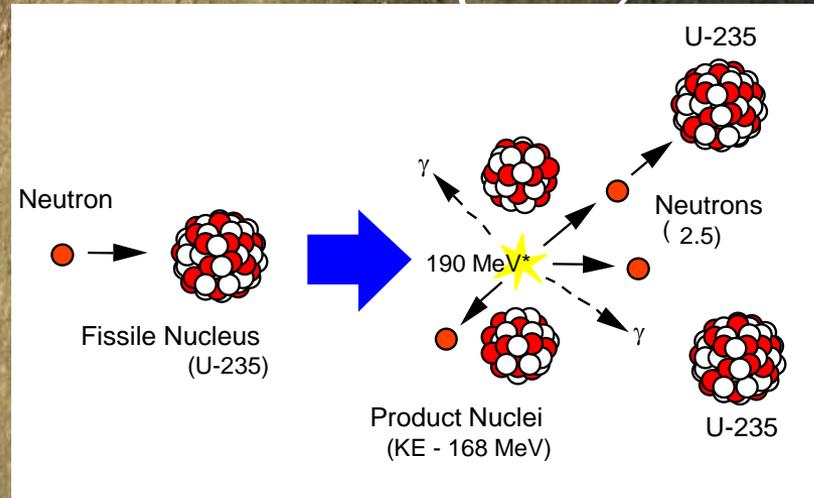


Heat Energy = 0.023 MeV/nucleon (0.558 W/g Pu-238)

Natural decay rate (87.7-year half-life)

- ◆ Long history of use on Apollo and space science missions
  - 44 RTGs and hundreds of RHUs launched by U.S. during past 4 decades
- ◆ Heat produced from natural alpha (α) particle decay of Plutonium (Pu-238)
- ◆ Used for both thermal management and electricity production

## Fission (U-235)



Heat Energy = 0.851 MeV/nucleon

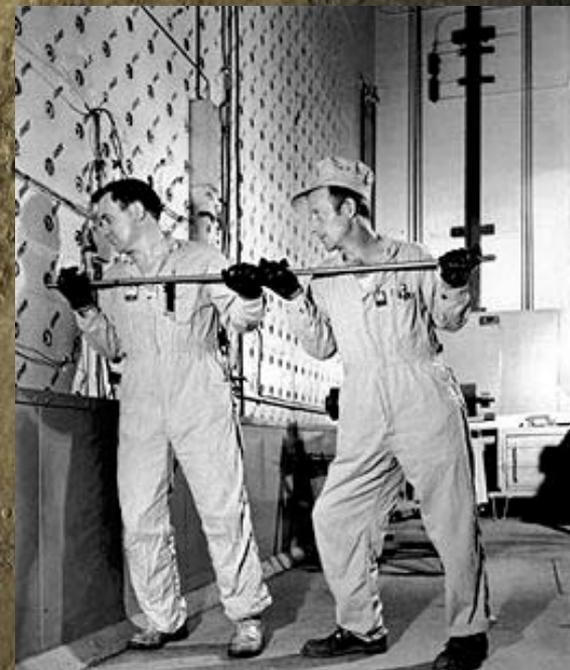
Controllable reaction rate (variable power levels)

- ◆ Used terrestrially for over 65 years
  - Fissioning 1 kg of uranium yields as much energy as burning 2,700,000 kg of coal
- ◆ One US space reactor (SNAP-10A) flown (1965)
  - Former U.S.S.R. flew 33 space reactors
- ◆ Heat produced from neutron-induced splitting of a nucleus (e.g. U-235)
  - At steady-state, 1 of the 2 to 3 neutrons released in the reaction causes a subsequent fission in a "chain reaction" process
- ◆ Heat converted to electricity, or used directly to heat a propellant



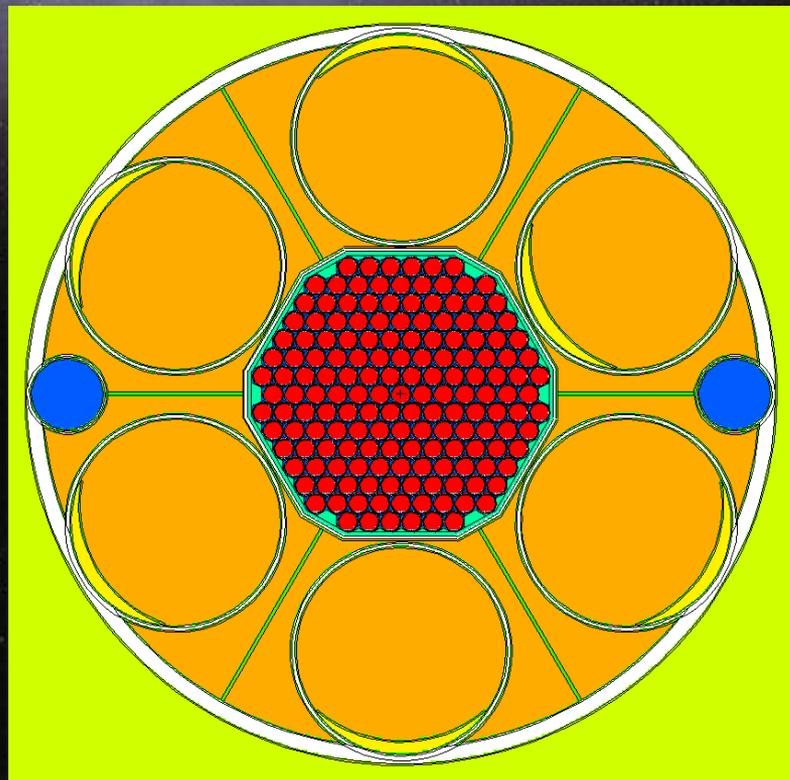
# Fission Introduction

- ◆ **Creating a fission chain reaction is conceptually simple**
  - Requires right materials in right geometry
- ◆ **Good engineering needed to create safe, useful, long-life fission systems**
  
- ◆ **1938 Fission Discovered**
- ◆ **1939 Einstein letter to Roosevelt**
- ◆ **1942 Manhattan project initiated**
- ◆ **1942 First sustained fission chain reaction (CP-1)**
- ◆ **1943 X-10 Reactor (ORNL), 3500 kWt**
- ◆ **1944 B-Reactor (Hanford), 250,000 kWt**
- ◆ **1944-now Thousands of reactors at various power levels**



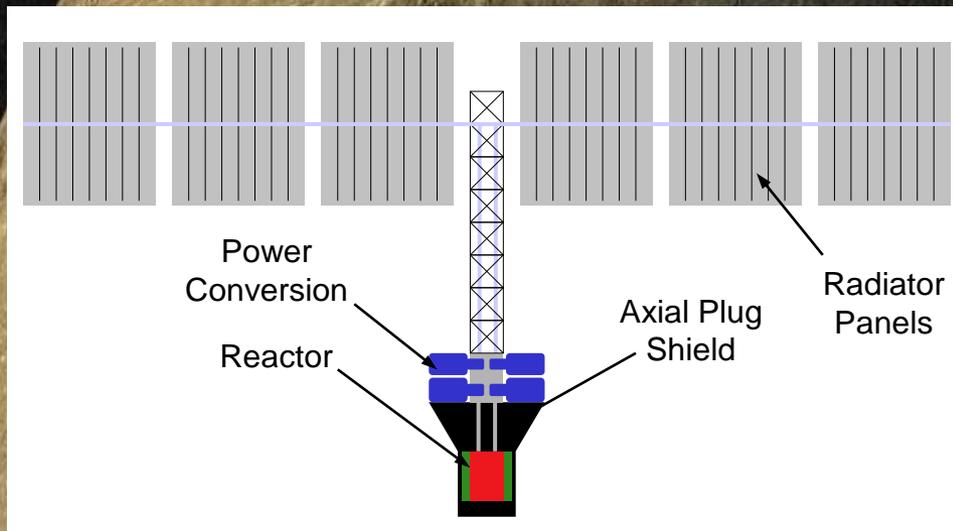


# Fission Reactor Operation



0.5 m

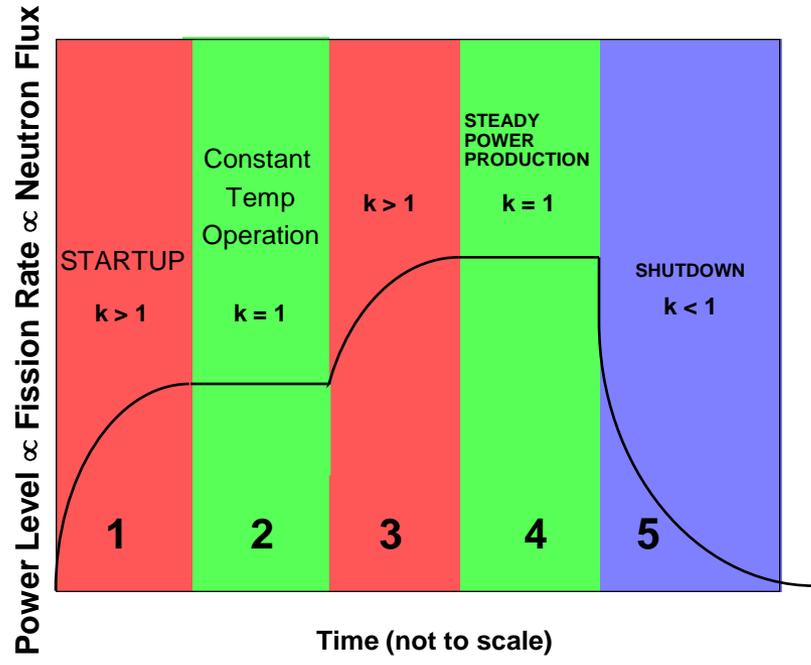
ARES 1



- ◆ System power controlled by neutron balance
- ◆ Average 2.5 neutrons produced per fission
  - Including delayed
- ◆ Constant power if 1.0 of those neutrons goes on to cause another fission
- ◆ Decreasing power if  $< 1.0$  neutron causes another fission, increasing if  $> 1.0$
- ◆ System controlled by passively and actively controlling fraction of neutrons that escape or are captured
- ◆ Natural feedback enables straightforward control, constant temperature operation
- ◆ 200 kWt system burns 1 kg uranium every 13 yrs



# Reactor Operation (Notional)



$k \equiv$  Multiplication Factor

$$= \frac{\text{Production Rate}}{\text{Loss Rate}} = \frac{N(t+l_n)}{N(t)}$$

$< 1$  (subcritical,  $dN/dt < 0$ )

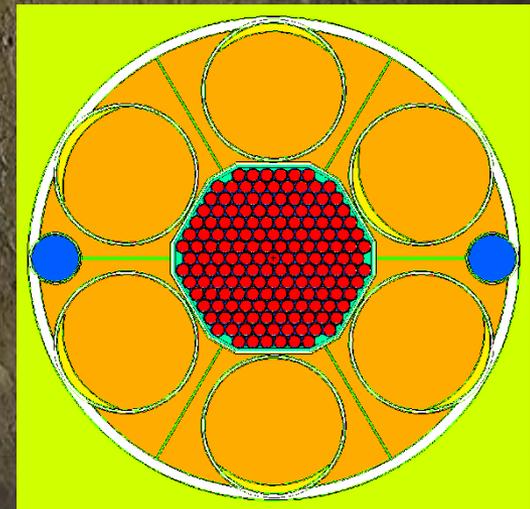
$= 1$  (critical,  $dN/dt = 0$ )

$> 1$  (supercritical,  $dN/dt > 0$ )

Thermal Power ( $t$ )  $\propto N(t)$

$$\text{Reactivity} \equiv \rho \equiv \frac{k-1}{k}$$

1. Control drums rotate to provide positive reactivity (supercritical). Power increases, reactor heats up.
2. As reactor temperature increases, natural feedback reduces reactivity to zero. System maintains temperature.
3. Control drums rotate to provide additional reactivity, until desired operating temperature is achieved.
4. Reactor follows load, maintaining desired temperature. Control drums rotate ~monthly to compensate for fuel that is consumed.
5. Control drums rotate to shut system down.





# Uranium Fuel

## ◆ Natural uranium consists of

- U-234      0.0055%
- U-235      0.720%
- U-238      99.274%

## ◆ Most reactor designs use uranium fuel enriched in U-235

## ◆ Prior to operation at power, uranium fuel is essentially non-radioactive and non-heat producing

## ◆ Following long-term operation, fission product decay power is 6.2% at $t=0$ (plus fission power from delayed neutrons)

- 1.3% at 1 hour
- 0.1% at 2 months

## ◆ Space reactor radiation exposure risk is primarily from inadvertent system start while personnel are near reactor

- Prevent inadvertent start via procedures, hardware, and design techniques developed over the past 6 decades



# Radiation Shielding

- ◆ **Reactor needs to be shielded during operation and for a period of time following operation at significant power**
- ◆ **Hydrogen bearing compounds (e.g. LiH, H<sub>2</sub>O) are most mass effective neutron shields**
  - Neutron shielding only needed while operating
- ◆ **High density, high atomic number materials (e.g. tungsten, uranium) best for gamma shielding, although areal density (mass/area) is primary requirement.**
- ◆ **NTP missions typically propose using propellant, consumables, and other “available” materials for shielding.**
- ◆ **Reactor can be shielded to any level desired**
  - Dose rate drops rapidly following shutdown



# Fission is Highly Versatile with Many Applications



Space Nuclear Power and Propulsion

## ◆ Small research reactors

- Examples include 2000 kWt TRIGA reactor recently installed in Morocco (< \$50M)

## ◆ Advanced, high-power research reactors and associated facilities

- Examples include the US Fast Flux Test Facility (400,000 kWt, ~\$3.0B FY08)

## ◆ Commercial Light Water Reactors 1,371,000 kWe (3,800,000 kWt)

- Recent TVA cost estimate ~\$2.2B

## ◆ Space reactors

- SNAP-10A 42 kWt / 0.6 kWe
- Soviet reactors typically 100 kWt / 3 kWe (some systems >150 kWt)
- Cost is design-dependent

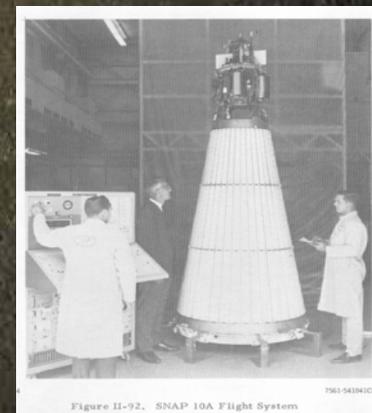
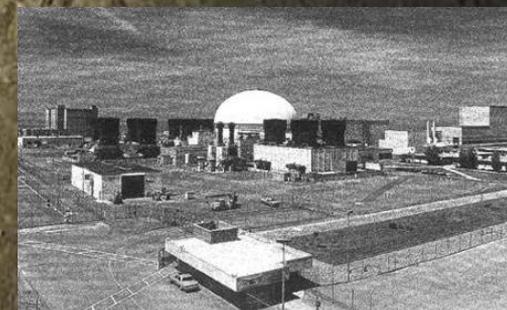


Figure II-92. SNAP 10A Flight System

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# Fission is Highly Versatile with Many Applications (continued)

## ◆ Naval Reactors

- Hundreds of submarines and surface ships worldwide

## ◆ Production of medical and other isotopes

## ◆ Fission Surface Power

- Safe, abundant, cost effective power on the moon or Mars

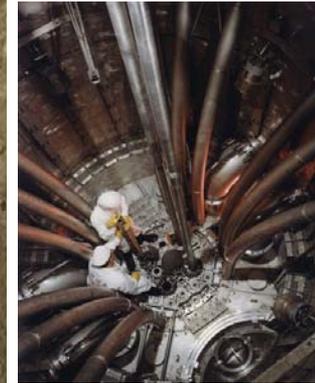
## ◆ Nuclear Thermal Propulsion

- Potential for fast, efficient transportation throughout inner solar system

## ◆ Nuclear Electric Propulsion

- Potential for efficient transportation throughout solar system

## ◆ Highly advanced fission systems for solar system exploration

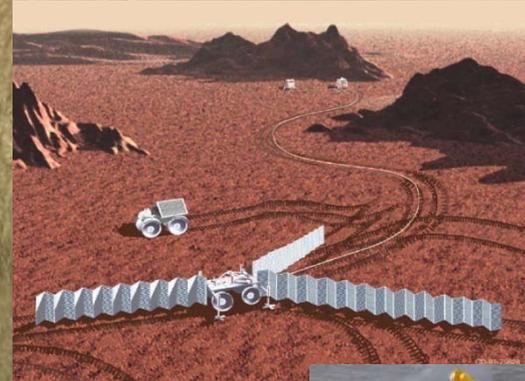




Space Nuclear Power and Propulsion

# Recent interest in Fission Surface Power (FSP) to support moon / Mars exploration

- ◆ **Continuous Day/Night Power for Robust Surface Operations**
- ◆ **Same Technology for Moon and Mars**
- ◆ **Suitable for any Surface Location**
  - Lunar Equatorial or Polar Sites
  - Permanently Shaded Craters
  - Mars Equatorial or High Latitudes
- ◆ **Environmentally Robust**
  - Lunar Day/Night Thermal Transients
  - Mars Dust Storms
- ◆ **Operationally Robust**
  - Multiple-Failure Tolerant
  - Long Life without Maintenance
- ◆ **Highly Flexible Configurations**
  - Excavation Shield Permits Near-Habitat Siting
  - Option for Above-Grade System or Mobile System (with shield mass penalty)
  - Option for Remote Siting (with high voltage transmission)
  - Option for Process Heat Source (for ISRU or habitat)





Space Nuclear Power and Propulsion

# Recent interest in Fission Surface Power (FSP) to support moon / Mars exploration

## ◆ Safe During All Mission Phases

- Launched Cold, No Radiation Until Startup
- Safe during Operation with Excavation or Landed Shield
- Safe after Shutdown with Negligible Residual Radiation

## ◆ Scalable to Higher Power Levels (kW to MW)

## ◆ Performance Advantages Compared to PV/RFC

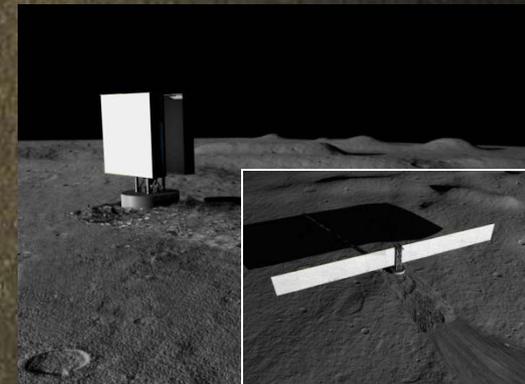
- Significant Mass & Volume Savings for Moon
- Significant Mass & Deployed Area Savings for Mars

## ◆ Competitive Cost with PV/RFC

- Detailed, 12-month “Affordable” Fission Surface Power System Cost Study Performed by NASA & DOE
- LAT2 FSP and PV/RFC Options had Similar Overall Cost
- Modest Unit Cost Enables Multiple Units and/or Multiple Sites

## ◆ Technology Primed for Development

- Terrestrial Reactor Design Basis
- No Material Breakthroughs Required
- Lineage to RPS Systems (e.g. Stirling) and ISS (e.g. Radiators, Electrical Power Distribution)





# “Affordable” Design Philosophy

## ◆ Conservative

- Low Temperature
- Known Materials and Fluids
- Generous Margins
- Large Safety Factors
- Terrestrial Design Basis

## ◆ Simple

- Modest Power & Life Requirements
- Simple Controls
  - Negative Temperature Reactivity Feedback: assures safe response to reactor temperature excursions
  - Parasitic Load Control: maintains constant power draw regardless of electrical loads and allows thermal system to remain near steady-state
- Slow Thermal Response
- Conventional Design Practices
- Established Manufacturing Methods
- Modular and Testable Configurations

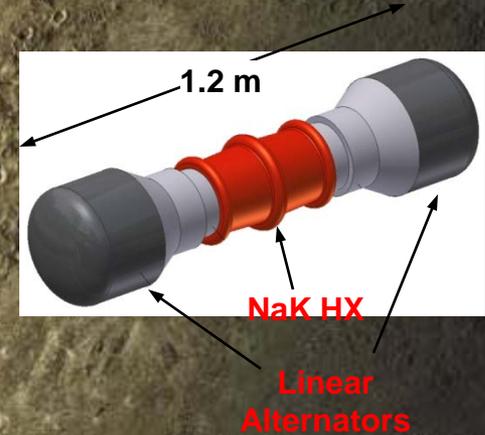
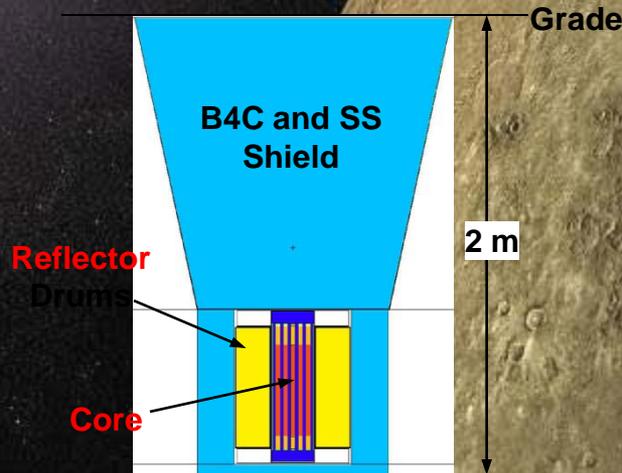
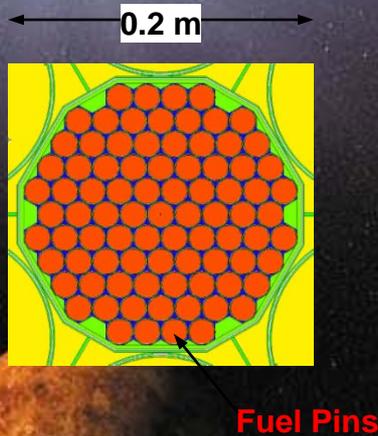
## ◆ Robust

- High Redundancy
- Fault Tolerance... including ability to recover from severe conditions such as:
  - Loss of Reactor Cooling
  - Stuck Reflector Drums
  - Power Conversion Unit Failure
  - Radiator Pump Failure
  - Loss of Radiator Coolant
  - Loss of Electrical Load
- High TRL Components
- Hardware-Rich Test Program
- Multiple Design Cycles

**Minimize Cost by  
Reducing Risk --  
Accept Mass Penalties  
if Needed**



# Key Design Features



## Reactor Core:

- ◆ Well-known  $\text{UO}_2$  fuel and SS-316 cladding at moderate temperature (<900K)
- ◆ Low power (<200 kWt), low fuel burn-up (~1%)
- ◆ Fluence levels well below material thresholds
- ◆ NaK coolant: low freeze temp (262K), extensive space & terrestrial technology base
- ◆ Simple and safe, negative temperature feedback control

## Reactor Module:

- Fault-tolerant, radial Be reflector control drums
- Low-risk B4C and SS shielding with regolith augmentation
- <2 Mrad and  $1 \times 10^{14}$  n/cm<sup>2</sup> at power conversion; <5 rem/yr at outpost (100 m)
- SS-316 primary & intermediate coolant loops with redundant EM pumps
- Cavity cooling with surface-mounted radiators

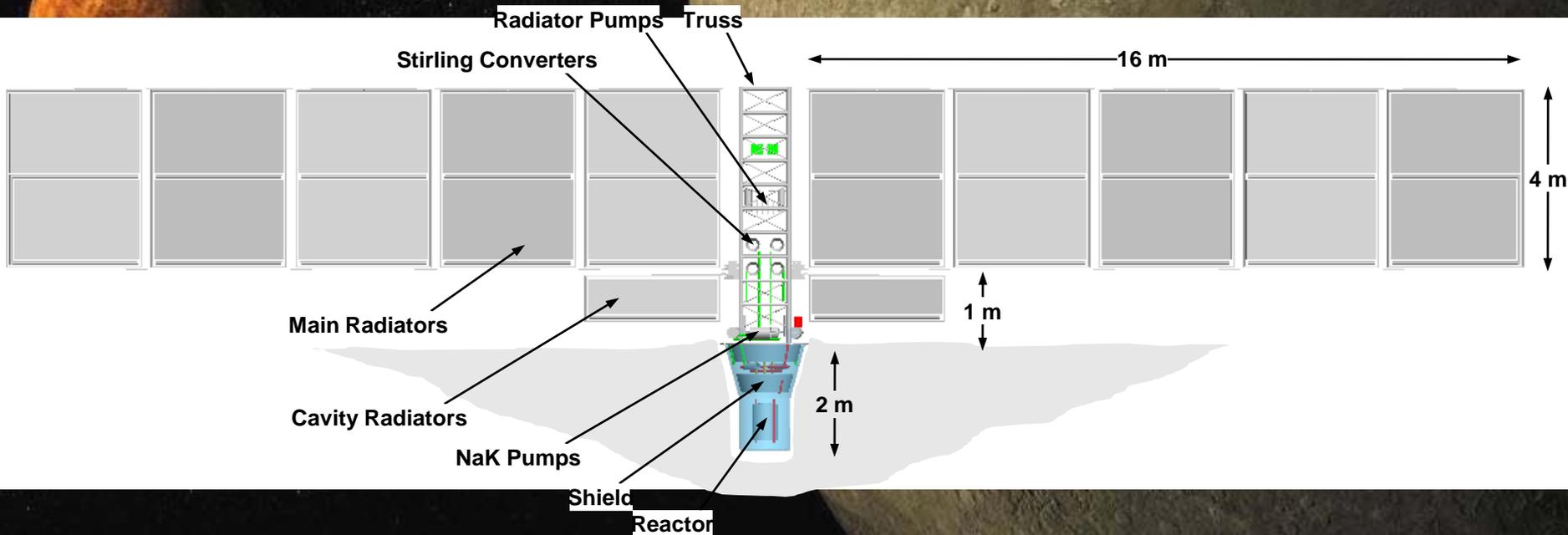
## Stirling Power Conversion:

- High efficiency (>25%) at low hot-end temperature (830K)
- Pumped-water cooling (400K)
- Smallest radiator size among PC options
- 4 dual opposed engines, 8 linear alternators
- 400 Vac power distribution
- Demonstrated technology at 25 kW size in 1980's
- Potential to leverage current RPS program



# FSP Reference Concept

- ◆ Modular 40 kWe System with 8-Year Design Life suitable for (Global) Lunar and Mars Surface Applications
- ◆ Emplaced Configuration with Regolith Shielding Augmentation Permits Near-Outpost Siting (<5 rem/yr at 100 m Separation)
- ◆ Low Temperature, Low Development Risk, Liquid-Metal (NaK) Cooled Reactor with  $UO_2$  Fuel and Stainless Steel Construction

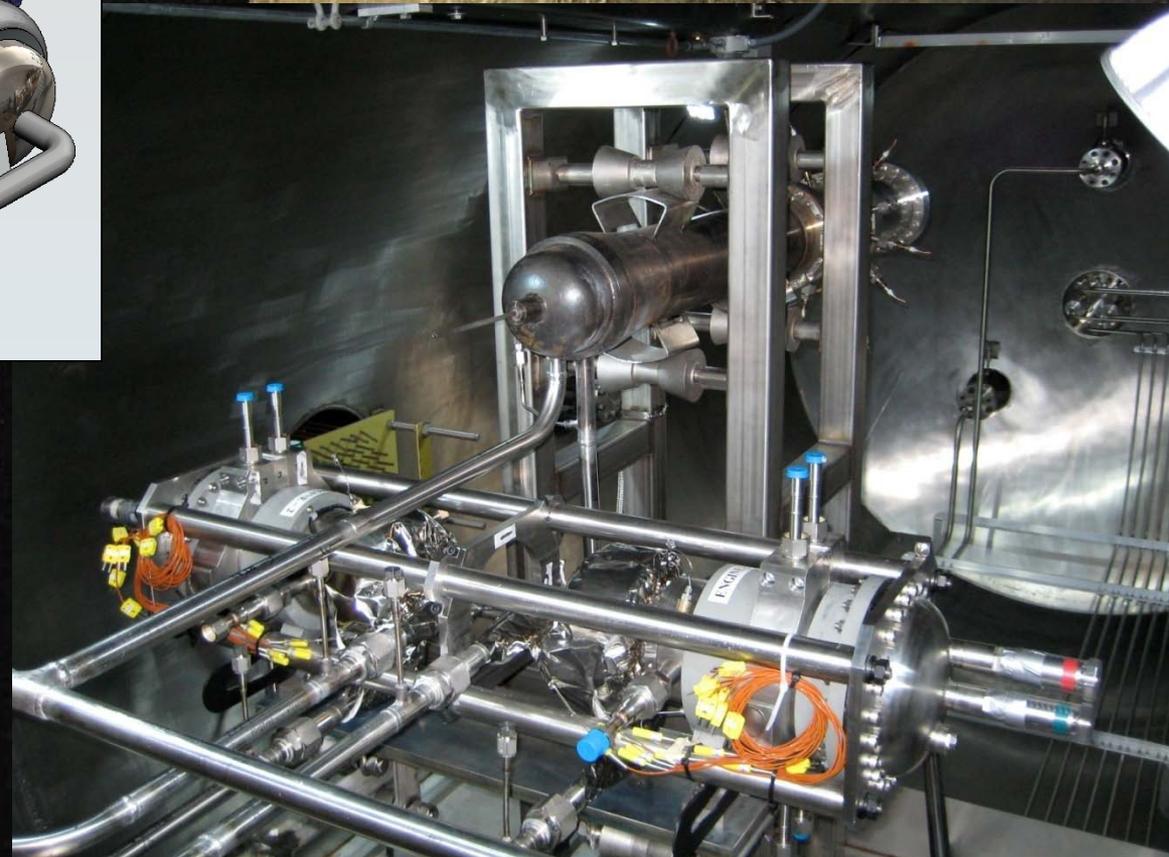
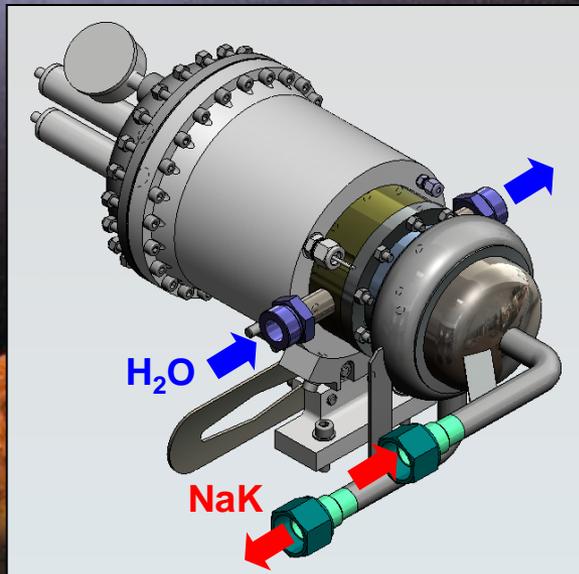




# 2 kWe NaK Stirling Demonstration Test

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Test Validated Reactor-Stirling  
Heat Transfer Approach for FSP  
(Stirling provided by NASA-GRC)



- 2.4 kWe at  
Thot=550°C,  
Tcold=50°C
- 32% Thermal  
Efficiency
- <5°C Circum. Gradient  
on Heater Head
- 41 Steady-State Test  
Points; 9 Transients
- 6 Reactivity Control  
Simulations



# Coupled NaK Loop / Stirling Test

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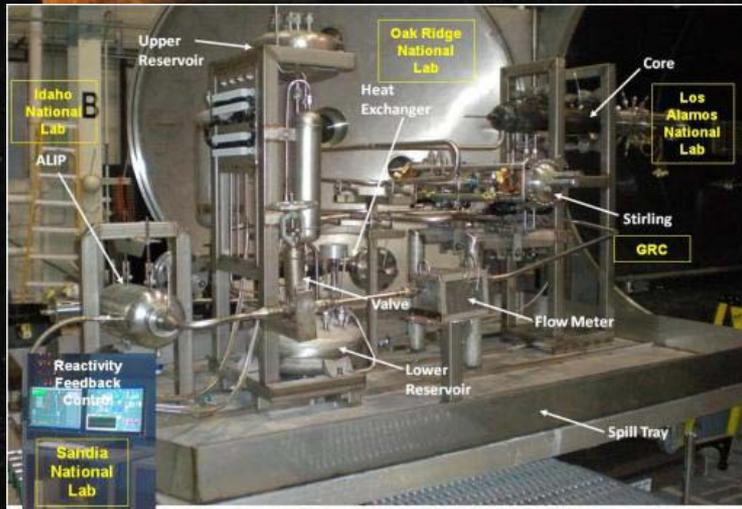
Cable tray providing protection from heat/NaK



Core Simulator Design by Los Alamos National Laboratory



Power Cable path to core



Integrated Stirling Test Assembly



ALIP Provided By Idaho National Laboratory



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# EFF-TF ALIP Test Circuit

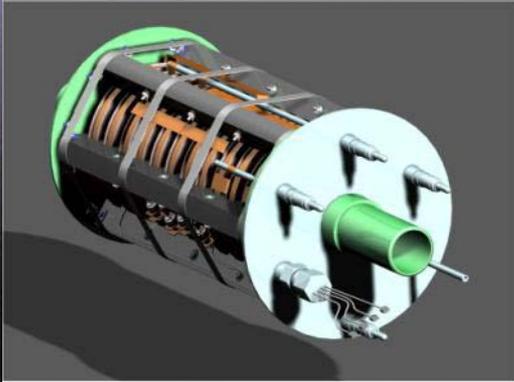
Performance Mapping of Annular Linear Induction Pump (ALIP) provided by Idaho National Laboratory





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# NaK Pump Testing



ALIP Drawing



ALIP unpacked at MSFC EFF-TF by INL and MSFC team members





# Performance Mapping of Annular Linear Induction Pump (ALIP) provided by Idaho National Laboratory

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## ALIP Test Circuit (ATC)



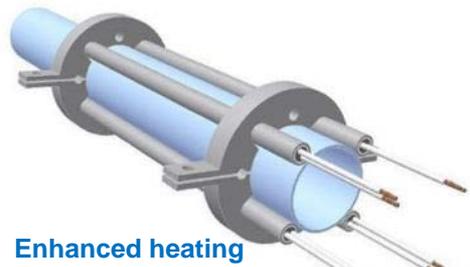
ALIP



ATC ready for chamber prior to NaK fill



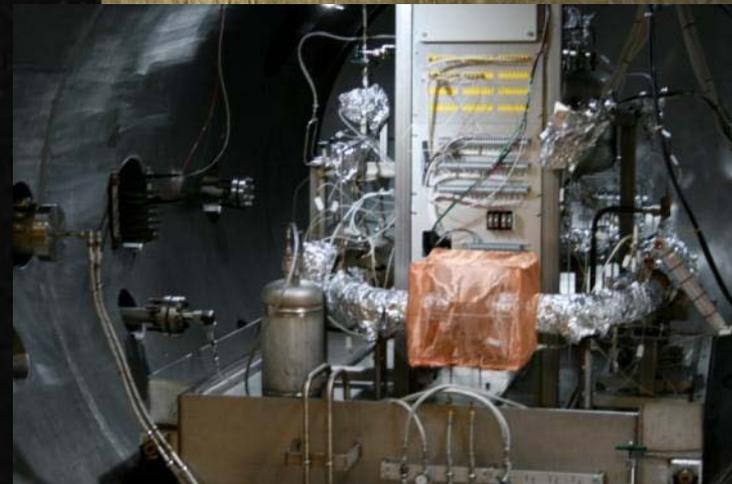
NaK fill



Enhanced heating assembly



Enhanced heating assembly ready for application of insulation



ATC Testing

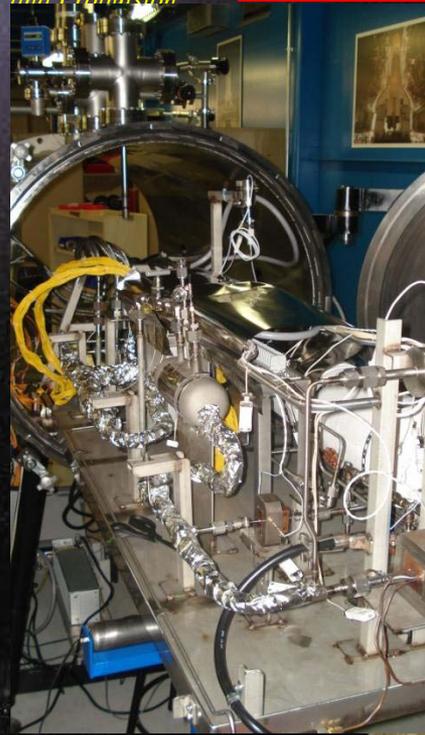




# EFF-TF Feasibility Test Loop



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**Feasibility Test Loop:**

**Investigate potential issues and optimizations related to pumped alkali metal systems**



# Fission Surface Power – Primary Test Circuit (FSP-PTC) 7 – Pin Reactor (Rx) Core Simulator Testing

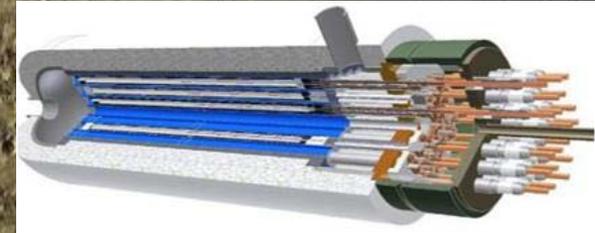
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MSFC  
Designed  
Advanced  
Simulators



7-Pin Rx  
Core Sim



37 – Pin TDU Rx Core Sim

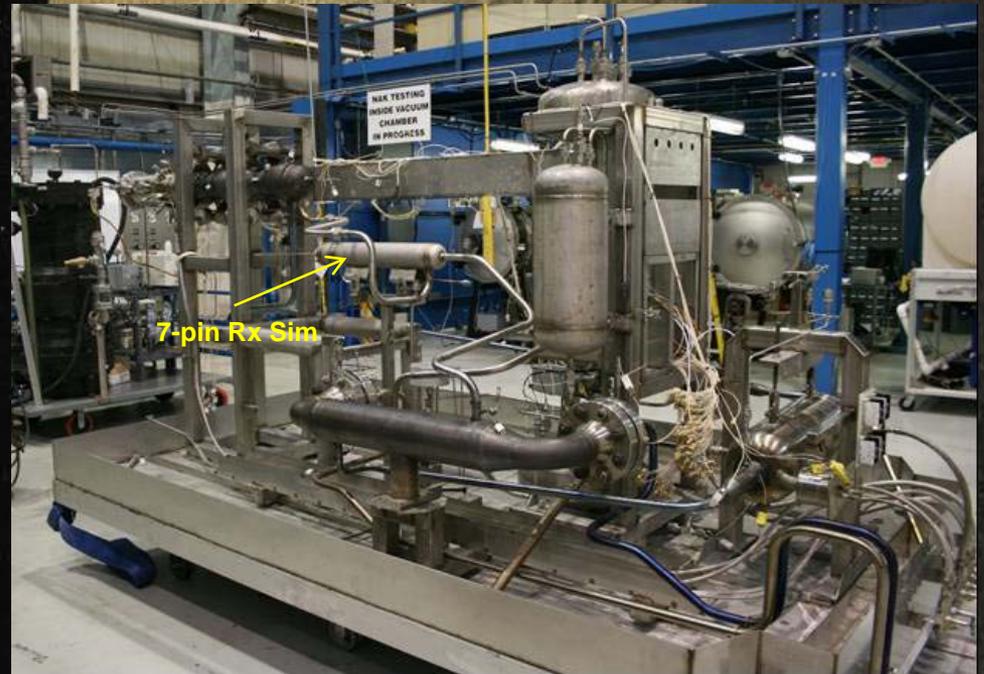


7 – Pin Rx Core Sim Rendering



7-pin Rx Sim

Revised FSP-PTC layout for 7 – Pin Rx Core Sim



7-pin Rx Sim

7 Pin Rx Core Sim installed in FSP-PTC



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# FSPS Accomplishments

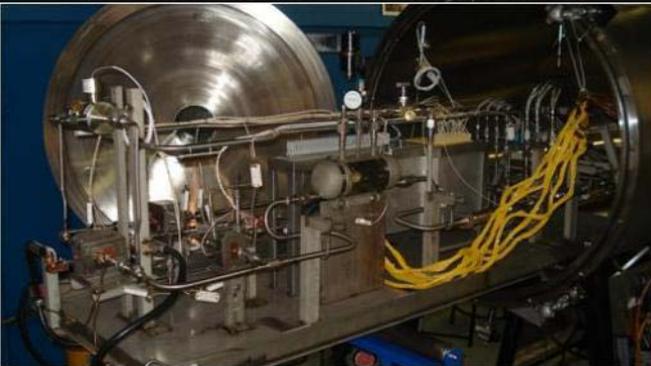
FSP-PTC  
Stirling &  
7 Pin Rx Core  
Sim  
Testing



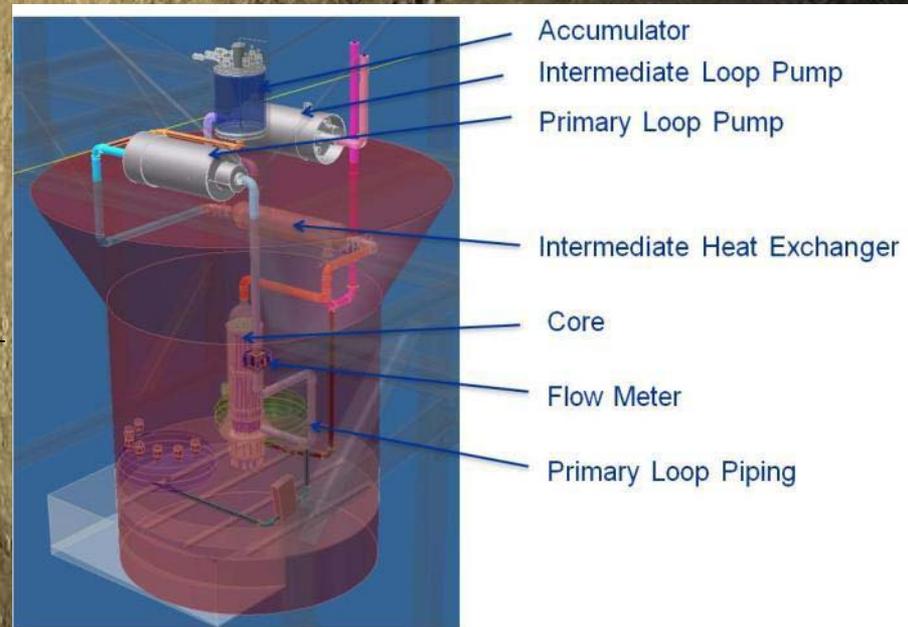
ATC  
Testing



FTL  
Testing



## Recent Activities Focused Towards TDU Reactor Simulator



MSFC Designed Reactor Simulator in TDU  
(top view close up)

**MILESTONES**  
Fabricate & Test : 2010-2011  
Ship to GRC 2012

# FSP Technology Project: Risk Reduction



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20 kWt NaK Reactor Simulator



NaK Annular Linear Induction Pump



25 kWe Dual Brayton System



2 kWe NaK Stirling System



10 kWe Stirling  
Alternator Test Rig



Ti-H<sub>2</sub>O Heat Pipe Life Test



2 kWe Direct Drive Gas Brayton



5 kWe Stirling Demonstrator

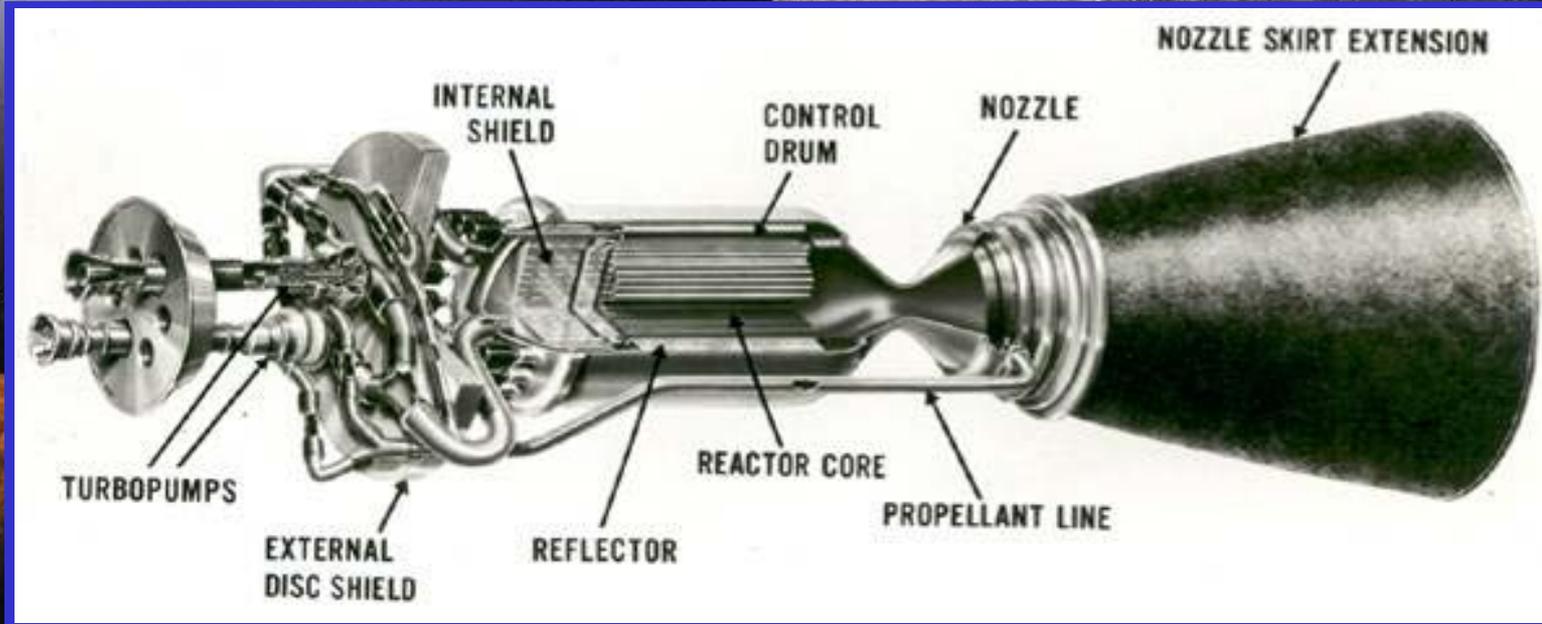


1 kWt Radiator  
Demo Unit





# Nuclear Thermal Propulsion (NTP)



- ◆ Typical system: hydrogen from propellant tank (not shown) directly heated by reactor and expanded through nozzle to provide thrust
- ◆ ~850 second Isp demonstrated in ground tests at high thrust/weight
- ◆ Potential for > 900 s Isp with advanced fuel forms and cycles
- ◆ Potential Applications
  - Rapid robotic exploration missions throughout solar system
  - Piloted missions to Mars and other potential destinations
  - Potential to significantly reduce propellant needs and/or trip time



# Nuclear Thermal Propulsion (NTP)



## ◆ NTP Concerns

- Cost/schedule – new engine system, nuclear testing, launch processing, potential opposition, INSRP process, etc.
- Potential operational constraints.

## ◆ NTP Benefits

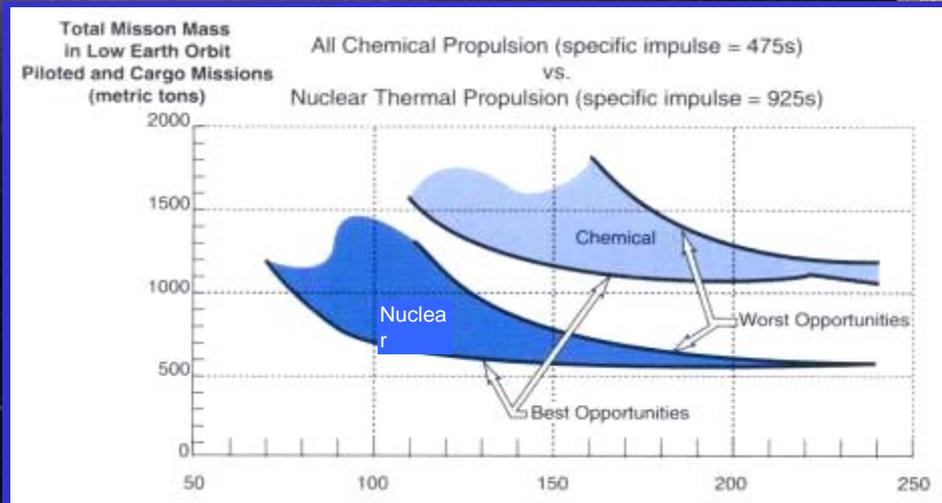
- Significant new capability. Reduce mission mass and/or time.
- Flexible choice of propellant, effectively unlimited energy.
- Significant cost savings /sustainable exploration program.



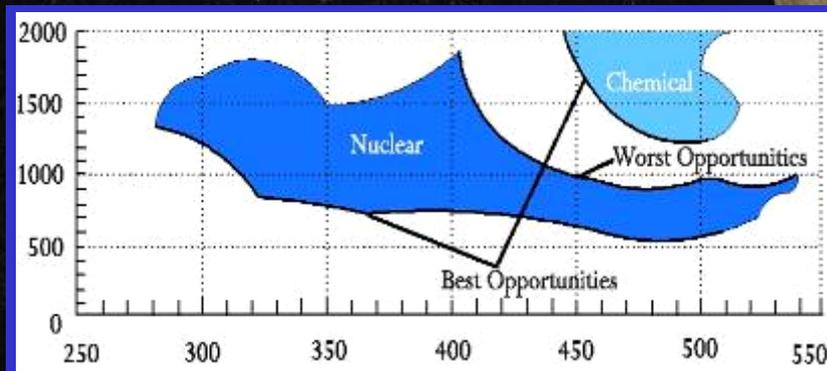
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# Nuclear Thermal Propulsion (NTP) Has The Potential to be Mission Enabling

## Comparison of IMLEO vs. Trip Time for All-up Opposition and Conjunction Mars Missions\*

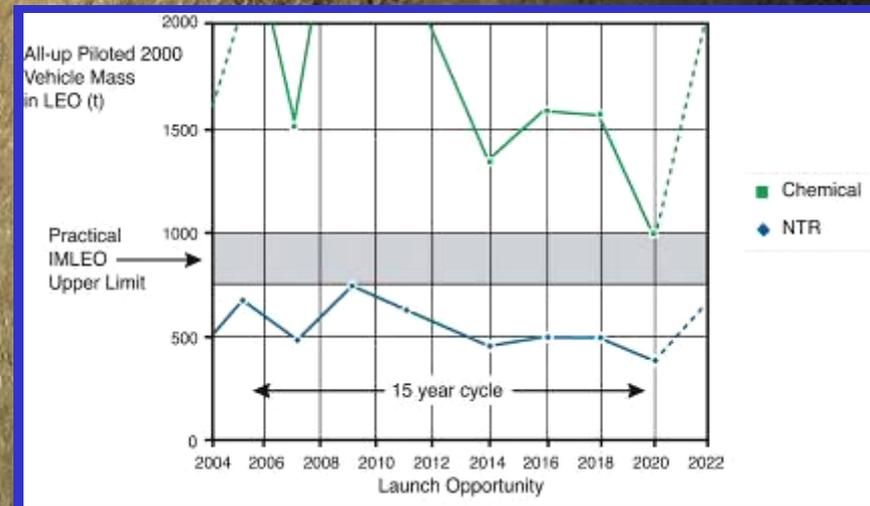


Conjunction Class (Long Stay) Mission



Opposition Class (Short Stay) Mission

Short Stay-Time Missions: NTP captures most opportunities, and chemical systems capture only one opportunity

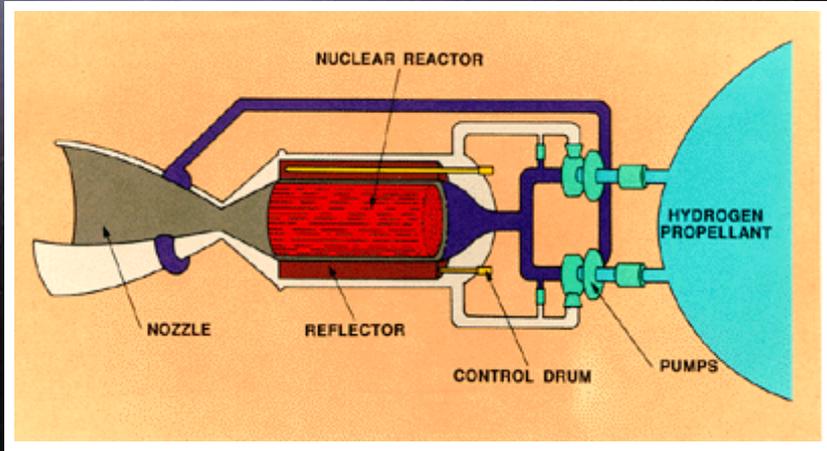


\*Source: NASA's Office of Aeronautics, Exploration and Technology, presented to Stafford Synthesis Team in 1991

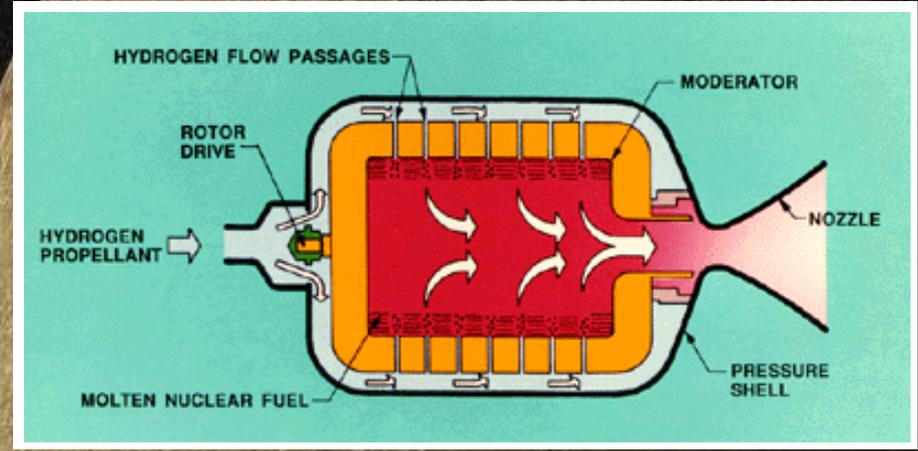


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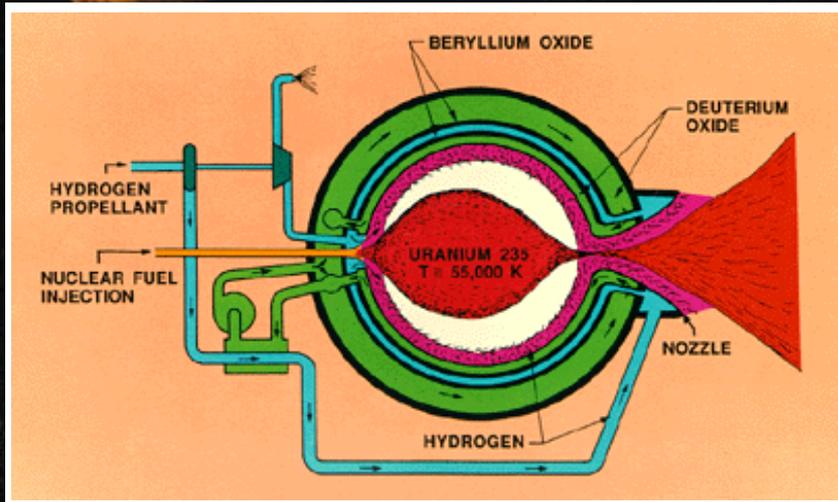
# Proposed Types of Nuclear Thermal Propulsion



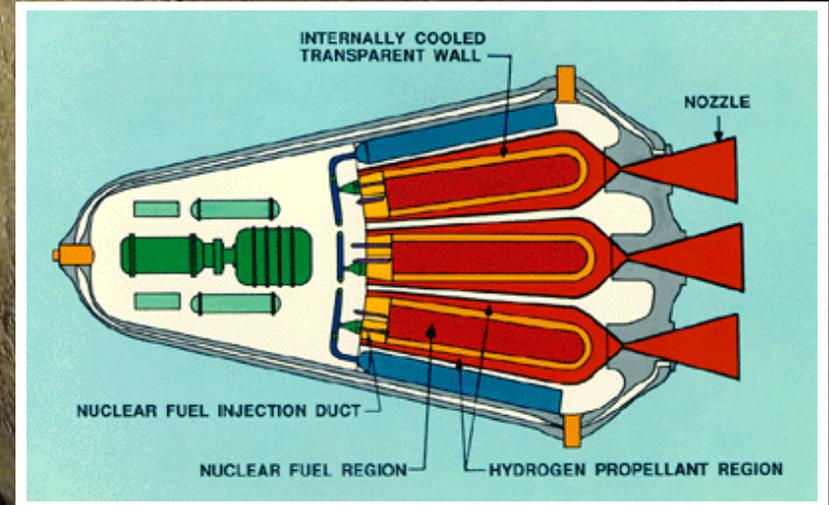
**SOLID CORE NUCLEAR ROCKET**



**LIQUID CORE NUCLEAR ROCKET**



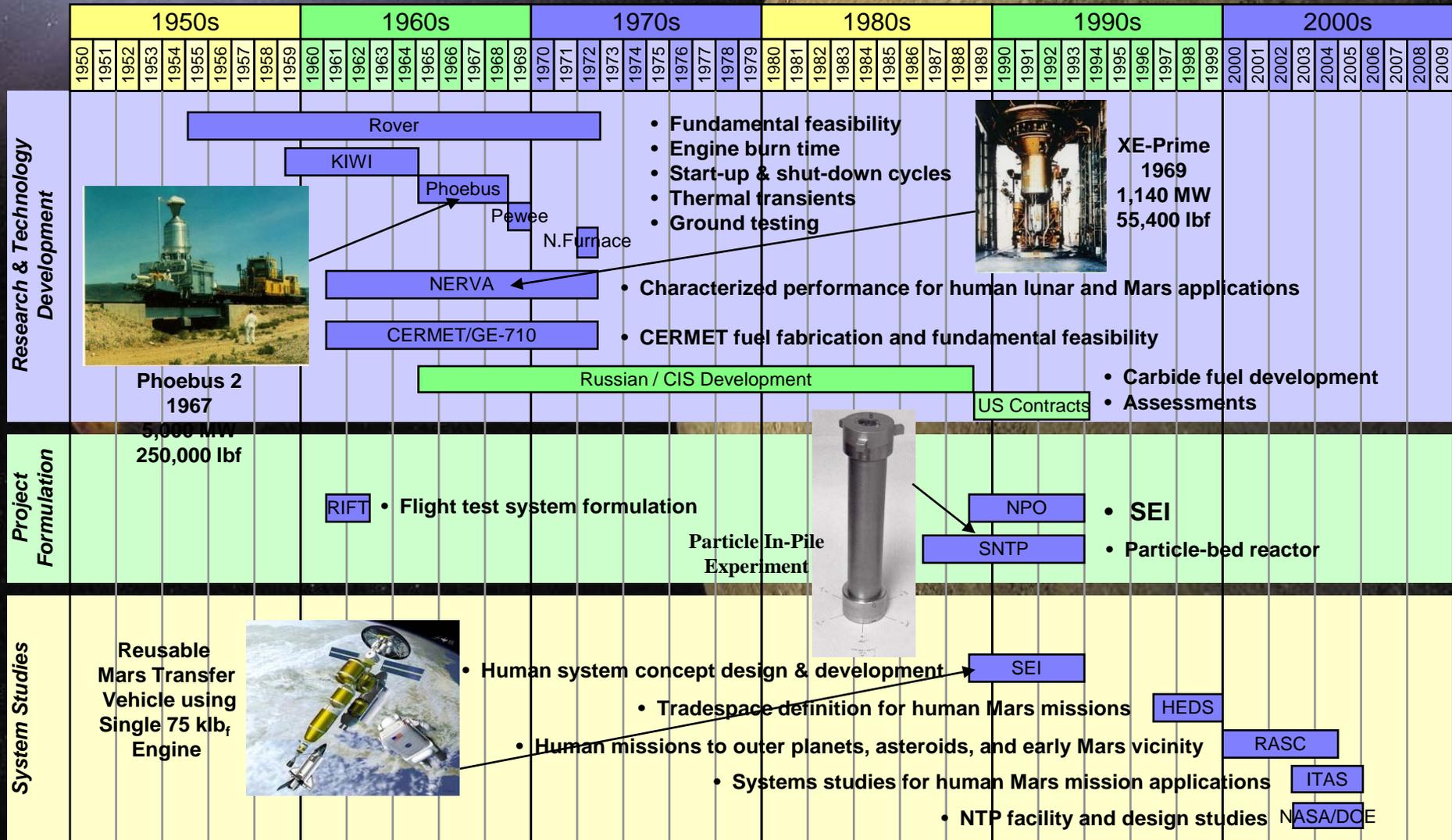
**Open-Cycle Gas Core Nuclear Rocket**



**Closed-Cycle Gas Core Nuclear Rocket**



# NTP History





# NTP could be mission-enhancing



NTP could enhance the ability to reach new destinations

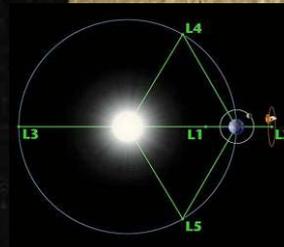
NTP could enable a steady, progressive, regular and affordable exploration program



Mars Cargo and Human Missions



Phobos Mission



Sun-Earth Lagrange Point



NEO Mission



Lunar Cargo Missions

As envisioned, NTP reduces required launch mass, reduces trip time, and increases mission opportunity. Over time, NTP could reduce exploration costs



# Rover/NERVA Engine Comparison

## Progression of Rover Reactors



KIWI A  
1958-1960  
100 MW  
0 lbf Thrust

KIWI B  
1961-1964  
1,000 MW  
50,000 lbf Thrust

Phoebus 1  
1965-1966  
1,000 & 1,500 MW  
50,000 lbf Thrust

Phoebus 2  
1967  
5,000 MW  
250,000 lbf Thrust

## Culmination of NERVA Program



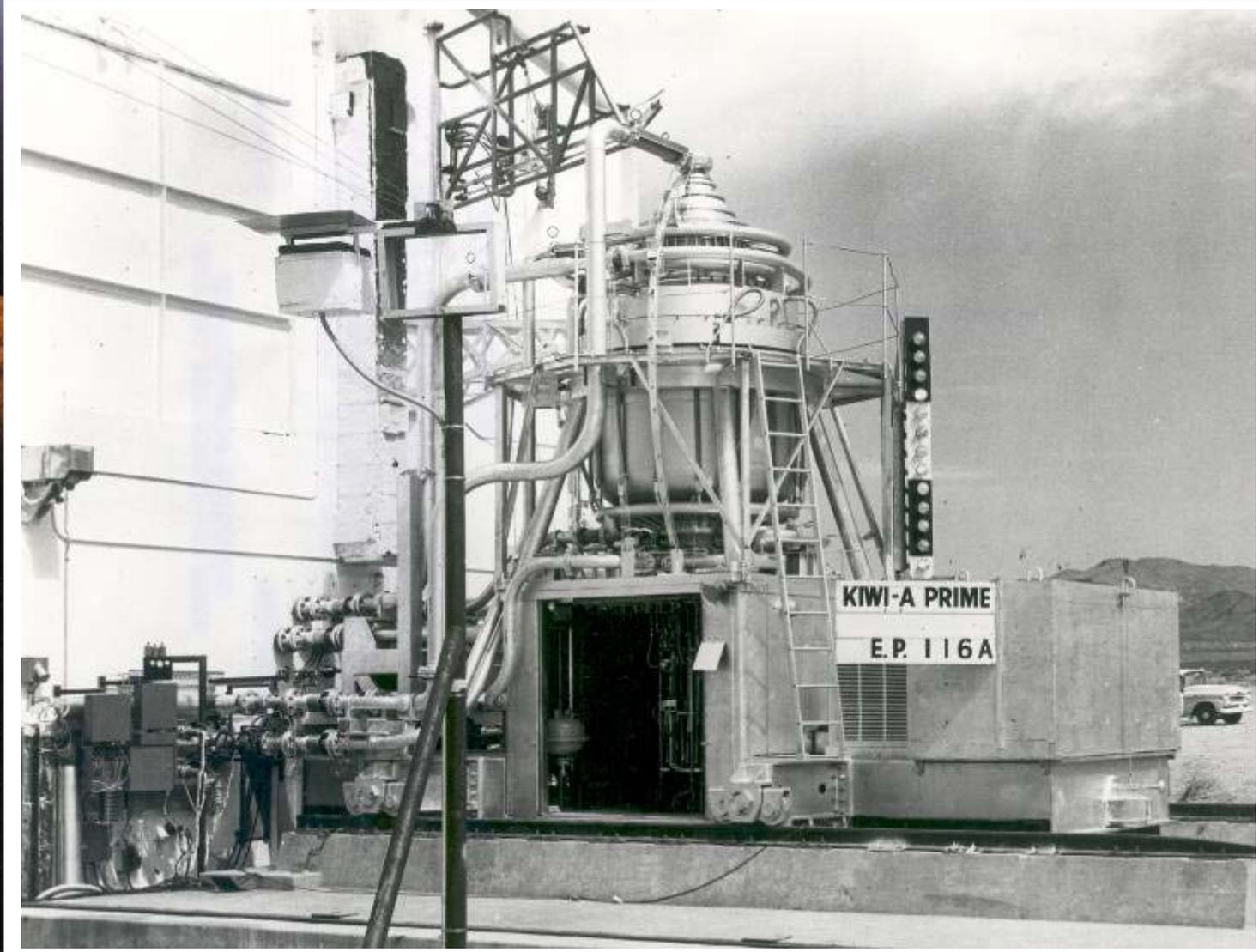
XE-Prime  
1969  
1,140 MW  
55,400 lbf Thrust



NERVA engines based largely on the KIWI B reactor design.

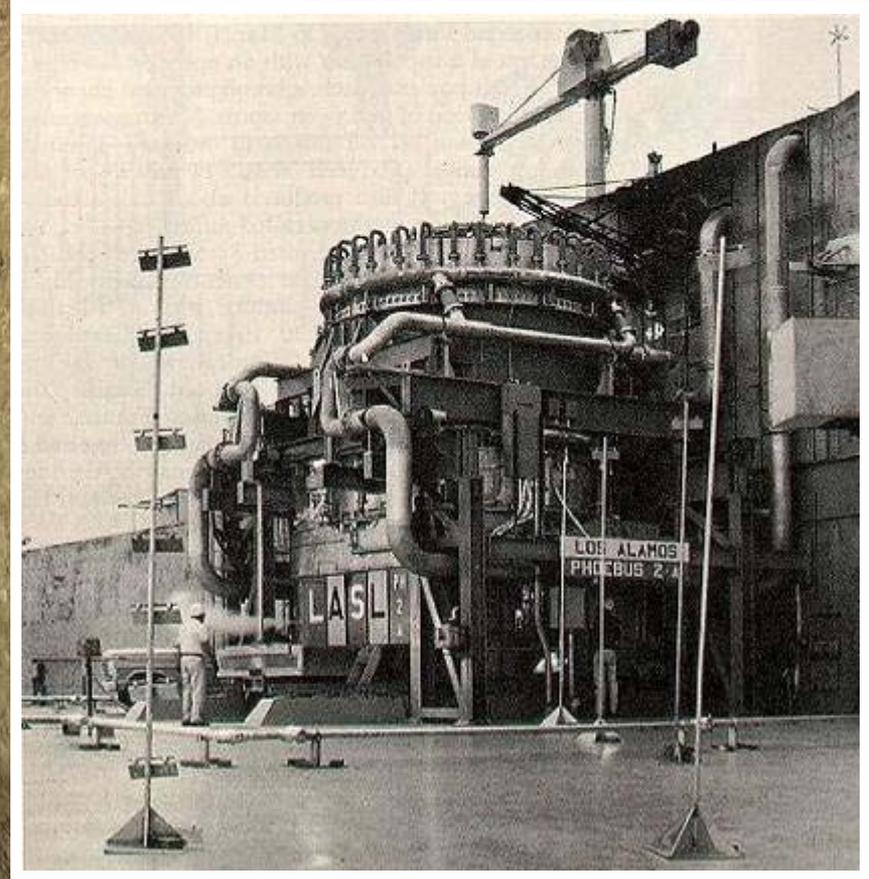
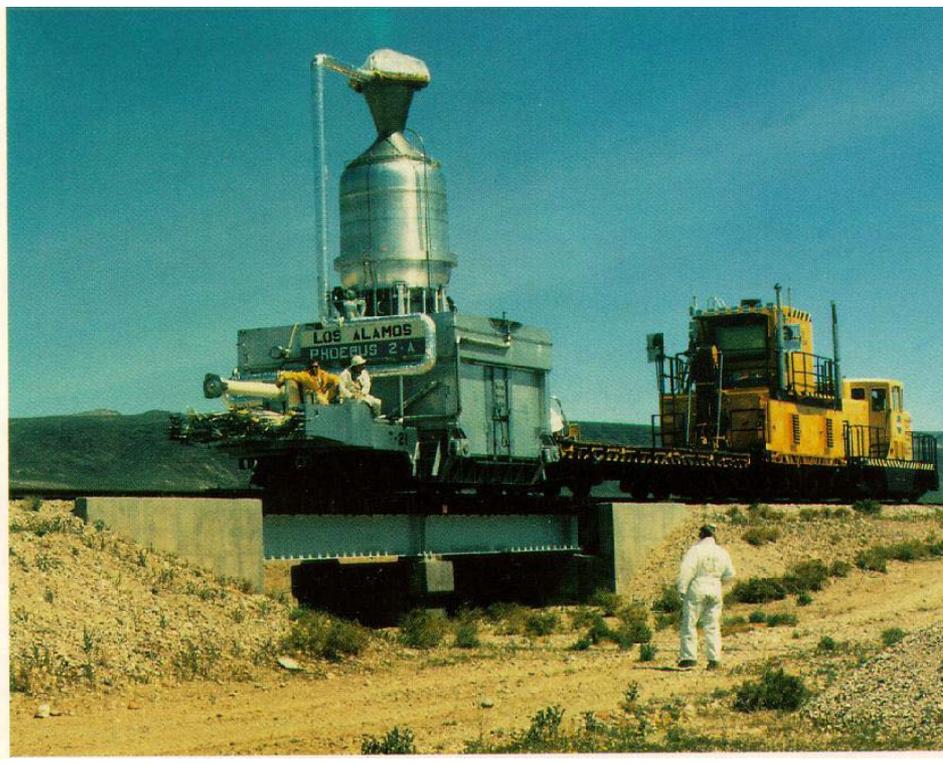


# KIWI A'

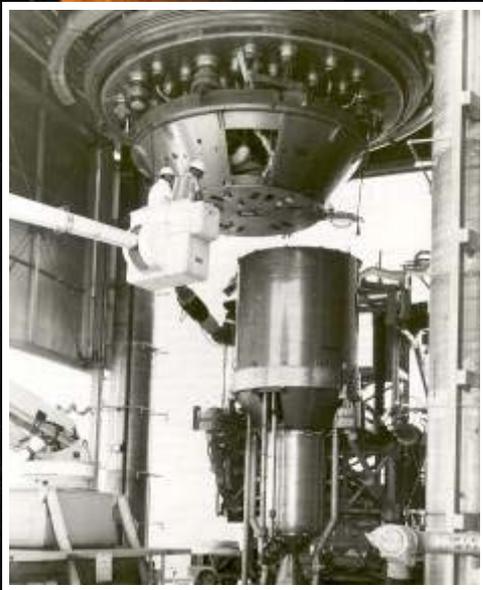
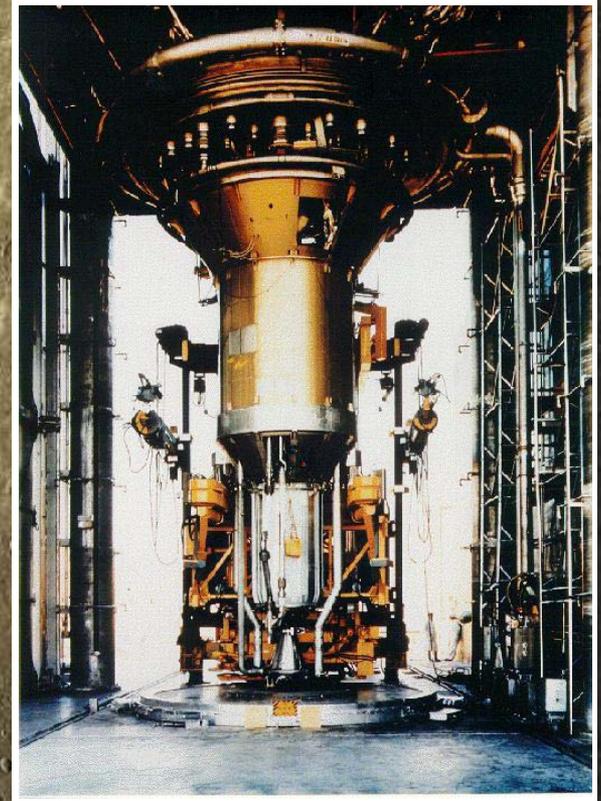
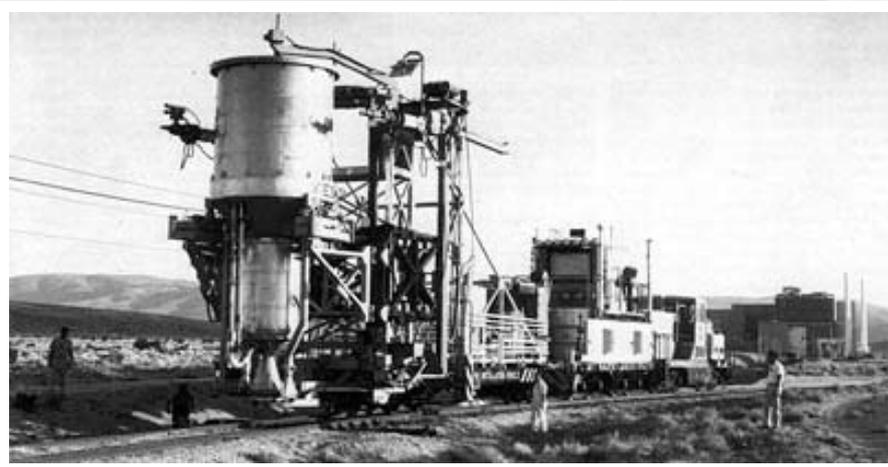




# Phoebus-2A



- ◆ Phoebus-2A
  - Tested 1968
  - 5 GW Reactor Core (tested at 4.2 GW)
  - 805 seconds Isp space Equiv.
  - 250,000 lbf Thrust



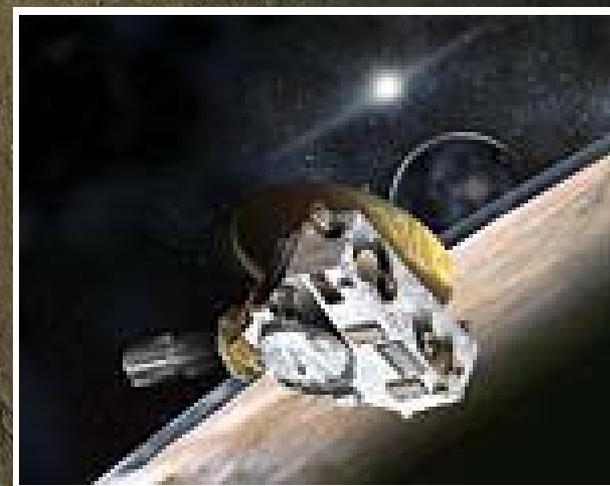
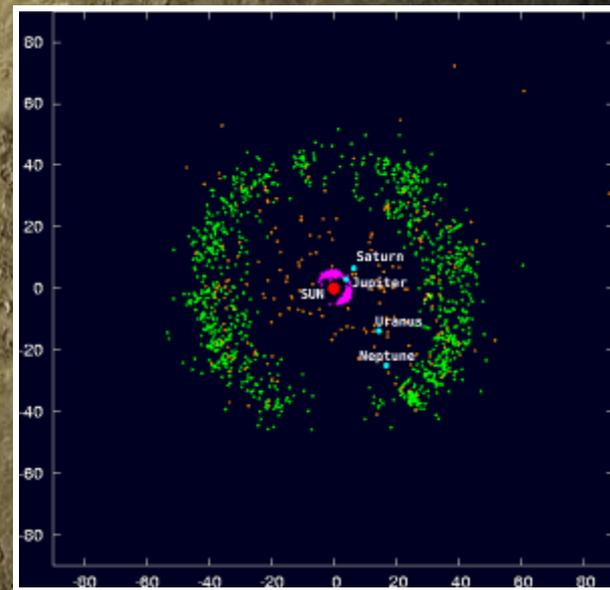
## ◆ XE' Engine

- Tested 1969
- 1.1 GW Reactor Core
- 820 seconds Isp space Equiv.
- 55,000 lbf Thrust



# Potential Advanced Topics - Example

- ◆ Over a thousand Kuiper Belt objects identified since 1992
  - Composed primarily of methane, ammonia, water
- ◆ Small icy moons, asteroids, and comets also identified
- ◆ Use nuclear thermal “steam” rockets to change orbits of icy bodies?
  - In theory, any vapor can be used for NTP propellant
  - No chemical reactions required
  - Improved NTP materials will improve performance
  - Gravity assists to reduce required  $\Delta V$
- ◆ Use icy bodies for propellant depots?
  - Volatiles used directly as propellant in NTP-based transportation system
- ◆ Use icy bodies for terraforming?

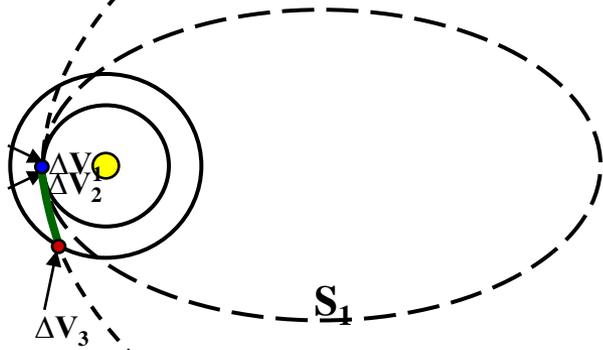




Space Nuclear Power and Propulsion

# Three-Burn Quick Mars Trip Quickest Mission w/o Becoming Hyperbolic

- Earth's Path
- Mars' Path
- Post  $\Delta V_1$  Ellipse
- Post  $\Delta V_2$  Ellipse
- Mars "Fast" Trajectory



$r_{\text{aphelion } 1} \approx 2.92 \text{ A.U.}$

$\Delta V_1$  (from LEO) = 5.01 km/s

$\Delta V_2$  (from  $S_1$  to  $S_2$ ) = 5.75 km/s

$\Delta V_3$  (from  $S_2$  to Mars) = 20.3 km/s

Payload: 100 mt

IMLEO: 1763.6 mt

$r_{\text{aphelion } 1} \approx 4.42 \text{ A.U.}$

$\Delta V_1$  (from LEO) = 5.96 km/s

$\Delta V_2$  (from  $S_1$  to  $S_2$ ) = 4.06 km/s

$\Delta V_3$  (from  $S_2$  to Mars) = 20.3 km/s

Payload: 100 mt

IMLEO: 1774.6

**1000 A.U. Ellipse is Near to a Solar System Escape Trajectory  
Time to Mars approx. 2.3 months**

$r_{\text{aphelion } 2} \approx 1000 \text{ A.U.}$

Larry Kos  
MSFC/TD31  
08/04/99

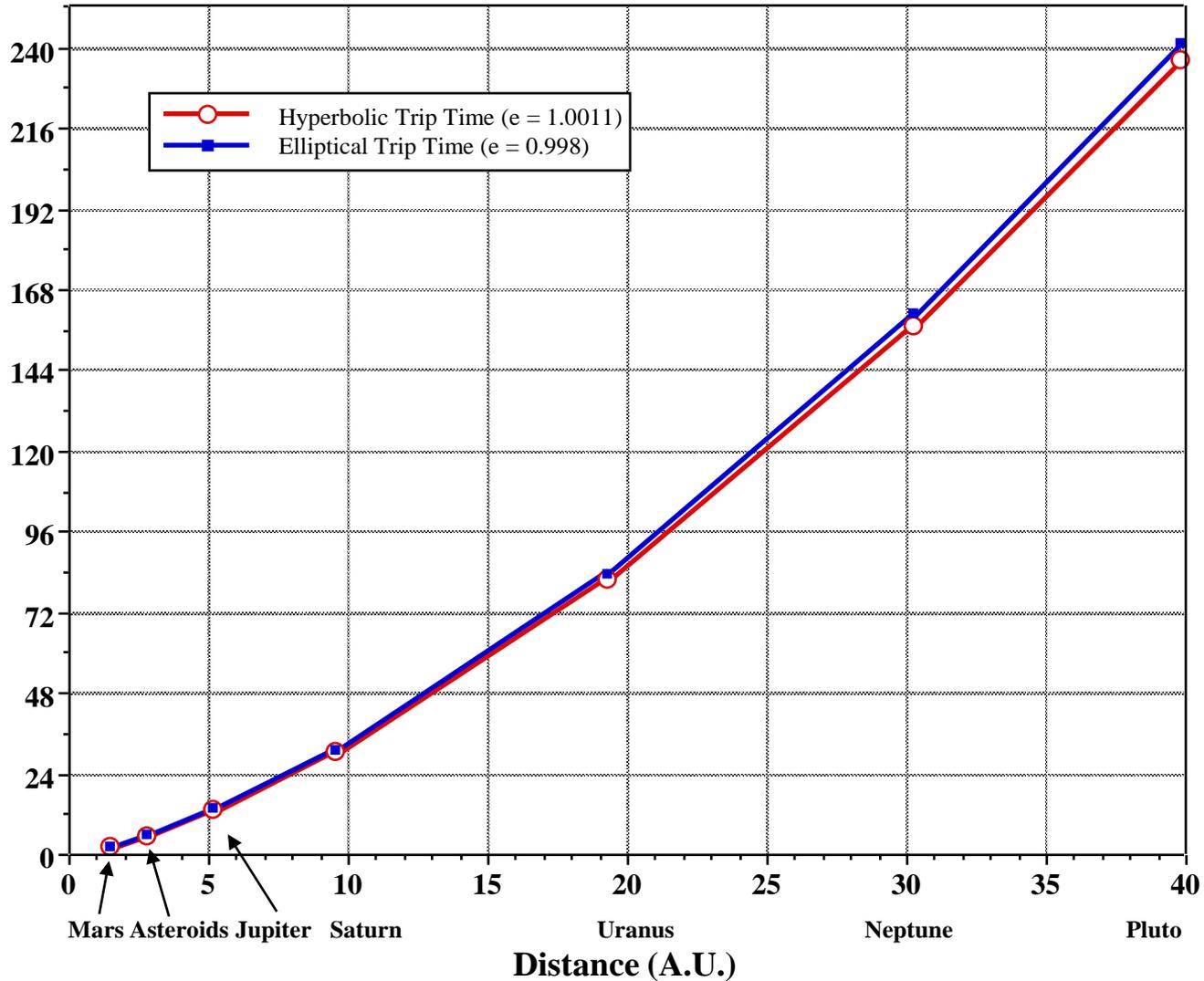


Space Nuclear Power and Propulsion

# Planetary Trip Times

## Quickest Missions w/o Becoming Hyperbolic

Spacecraft  
Trip Time,  
one-way  
(30 days = 1 unit)





# Beyond Fission: Potential Futuristic Nuclear Energy Sources

Space Nuclear Power and Propulsion

## Fusion Reactions

### Sun

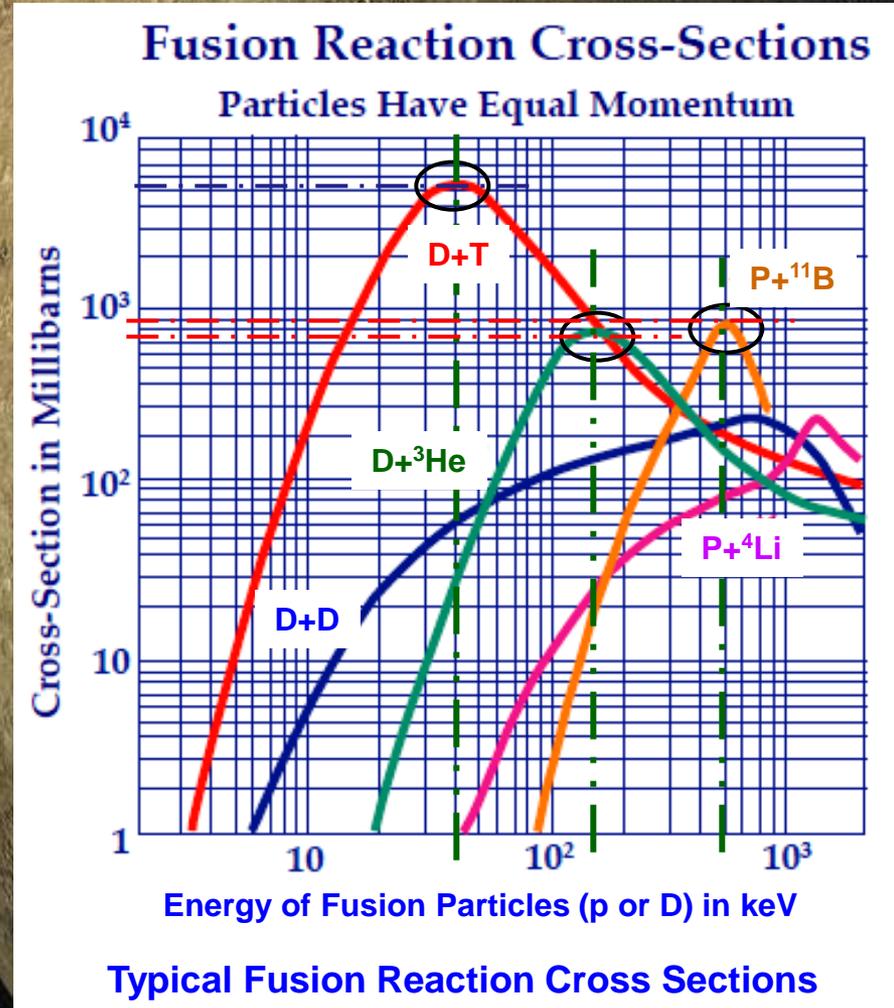
- $1\text{H} + 1\text{H} \rightarrow 2\text{H} + \text{antielectron} + \text{neutrino}$
- $1\text{H} + 1\text{H} \rightarrow 2\text{H} + \text{antielectron} + \text{neutrino}$
- $\text{electron} + \text{antielectron} \rightarrow \text{photon} + \text{photon}$
- $\text{electron} + \text{antielectron} \rightarrow \text{photon} + \text{photon}$
- $2\text{H} + 1\text{H} \rightarrow 3\text{He} + \text{photon}$
- $2\text{H} + 1\text{H} \rightarrow 3\text{He} + \text{photon}$
- $3\text{He} + 3\text{He} \rightarrow 4\text{He} + 1\text{H} + 1\text{H}$

### Net Result:

$$4\text{ }^1\text{H} + 2\text{e} \Rightarrow \text{}^4\text{He} + 2\text{ neutrinos} + 6\text{ gamma (26 MeV)}$$

### Potential Small, Controlled Systems

- $\text{D} + \text{T} \Rightarrow \text{n}^0\text{ (14.07 MeV)} + \text{}^4\text{He (3.52 MeV)}$
- $\text{D} + \text{D} \Rightarrow \text{n}^0\text{ (2.45 MeV)} + \text{}^3\text{He (0.82 MeV)}\text{ (50\%)}$
- $\text{D} + \text{D} \Rightarrow \text{p (3.02 MeV)} + \text{T (1.01 MeV)}\text{ (50\%)}$
- $\text{D} + \text{}^3\text{He} \Rightarrow \text{p (14.68 MeV)} + \text{}^4\text{He (3.67 MeV)}$
- $\text{}^3\text{He} + \text{}^3\text{He} \Rightarrow \text{}^4\text{He} + 2\text{ p (12.9 MeV)}$
- $\text{p} + \text{}^{11}\text{B} \Rightarrow 3\text{ }^4\text{He (8.7 MeV)}$





# Beyond Fission: Potential Futuristic Nuclear Energy Sources

Space Nuclear Power and Propulsion

**Fusion:** The performance potential of lightweight, high gain fusion propulsion systems operating with aneutronic fuels (e.g.  $p\text{-}^{11}\text{B}$ ) theoretically exceeds that of fission by an order of magnitude.

## Fundamental Issues to Resolve:

**1. Aneutronic Fuels.** The performance potential of fusion propulsion systems operating with deuterium or tritium bearing fuels (e.g. D-T, D-D, or D- $^3\text{He}$ ) is severely limited because of waste heat production from neutron kinetic energy, and the additional waste energy released when a neutron of any energy is captured. The use of aneutronic fuels (e.g.  $p\text{-}^{11}\text{B}$ ) will be required for high performance.

**2. High Gain.** Recent studies (Chakrabarti et al., 2001) have shown that high engineering gain ( $Q > 50$ ) is needed to minimize the mass of the fusion reaction driver and enable high performance.

**3. Compact Systems.** Significant funds and five decades have been spent on research related to controlled fusion. While the two leading approaches for achieving engineering breakeven are extremely massive, knowledge and experience from the ongoing terrestrial fusion effort may be useful in devising compact systems suitable for space propulsion applications.

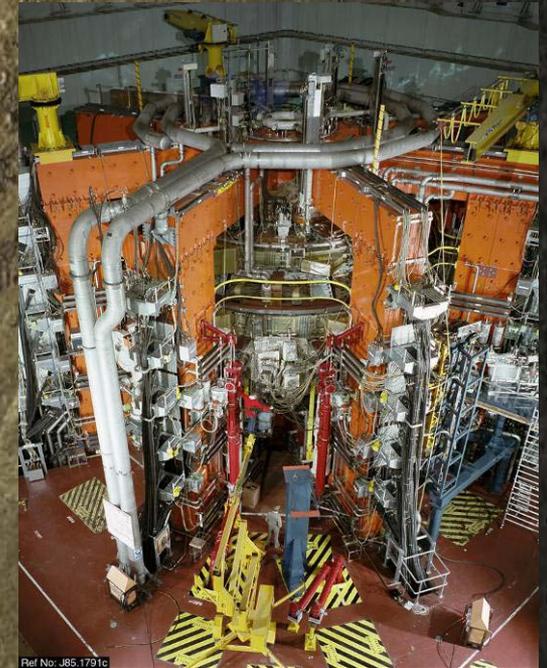
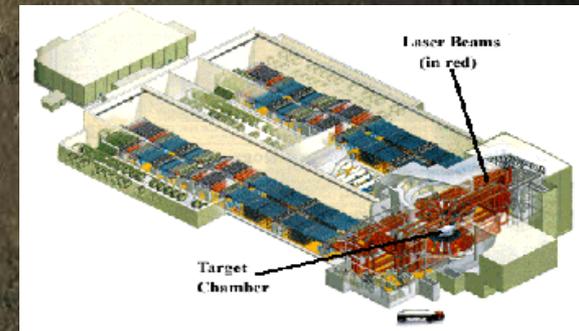


Photo Courtesy of EFDA-JET



National Ignition Facility



Space Nuclear Power and Propulsion

# Beyond Fission: Potential Futuristic Nuclear Energy Sources

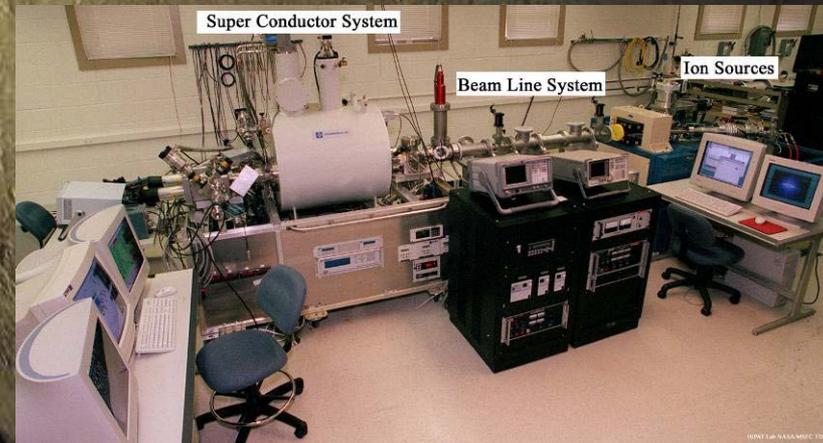
**Antimatter:** Energy stored as antimatter has a specific energy of  $1.8 \times 10^{17}$  J/kg, over 500 times that of fission or fusion.

## Fundamental Issues to Resolve:

- 1. Production.** Antiproton production rates must increase by several orders of magnitude, and the cost per antiproton must decrease correspondingly.
- 2. Storage.** Effective methods for long-term antiproton storage and transportation must be developed.
- 3. Thrust Production.** Effective methods for converting energy stored as antimatter into high specific impulse thrust must be devised.



Antiproton Decelerator at CERN



High Performance Antiproton Trap (HiPAT) at NASA MSFC

Photo: Courtesy of CERN

Photo: Marshall Space Flight Center



# Summary

- ◆ Nuclear power and propulsion systems can enable exciting space exploration missions. These include bases on the moon and Mars; and the exploration, development, and utilization of the solar system.
- ◆ In the near-term, fission surface power systems could provide abundant, constant, cost-effective power anywhere on the surface of the Moon or Mars, independent of available sunlight. Affordable access to Mars, the asteroid belt, or other destinations could be provided by nuclear thermal rockets.
- ◆ In the further term, high performance fission power supplies could enable both extremely high power levels on planetary surfaces and fission electric propulsion vehicles for rapid, efficient cargo and crew transfer. Advanced fission propulsion systems could eventually allow routine access to the entire solar system. Fission systems could also enable the utilization of resources within the solar system. Fusion and antimatter systems may also be viable in the future.